Applications of Accelerators to Environmental Protection at the Idaho Accelerator Center

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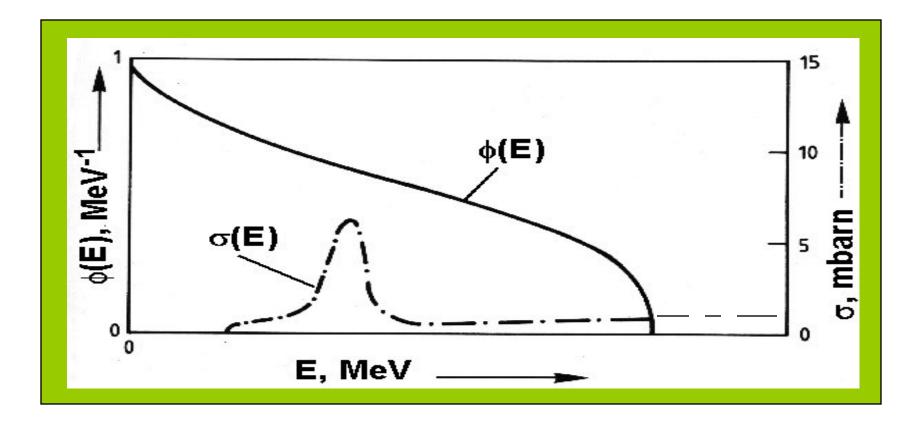
Applications of Photon Activation and Activation Analysis (PAA) (with environmental implications)

- Trace element analysis of waste streams,
- Trace element analysis of environmental samples (air, water/ice, soil/rock, biological),
- Isotope Production and the avoidance of nuclear waste,
- 'Burnup' of nuclear waste with 'coupled' accelerator-reactor systems.

Why PAA? <u>Sensitivity and Non-Destructivity:</u> Absolute Sensitivity – ignoring "matrix effects", Relative Sensitivity in most cases is a few ppb.

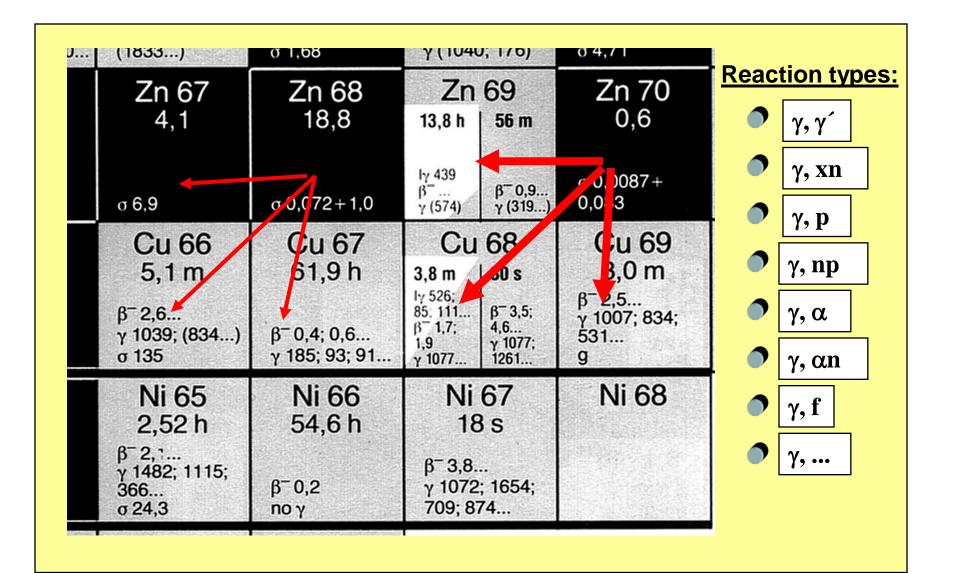
Element	Sensitivity µg
Ν	0.02
0	0.05
F	0.001
CI	0.01
Ni	0.6
Sr	0.02
Cd	0.2
Au	0.02
ТІ	0.4
Pb	1.0
U	0.0005

How does it work? Yield = Radionuclide production rate (neglecting decay)



$$Y \propto M \int_{E_S}^{E_{\max}} \varphi(E_{\gamma}) \cdot \sigma(E_{\gamma}) dE_{\gamma}$$

The general principle



Example: Waste Assay

• Why PAA?

• What are its advantages?

• Is it practical?



The Problem of Large Amounts in chemical analysis

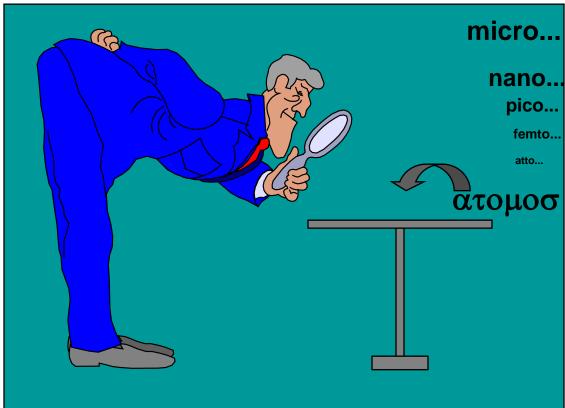
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The current "modern trends" in chemical analysis...



milli...



Almost ignored, up to present:



The characterisation of large amounts / volumes of material

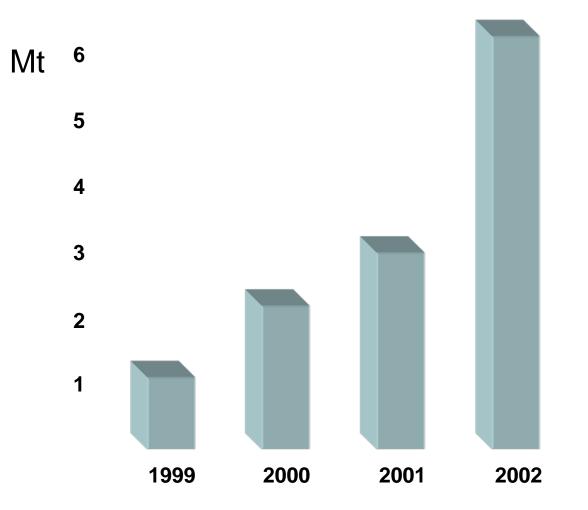


Example: electronic waste; catalysts





Example: electronic waste in Germany



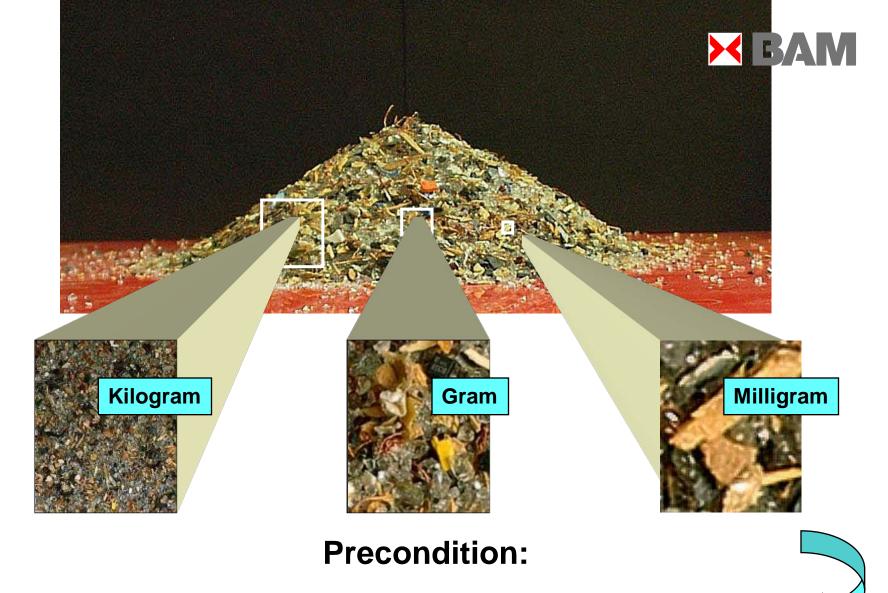


Hazardous:

- Chromium
- Nickel
- Arsenic
- Bromine
- Cadmium
- Lead
- Bismuth
- etc.

Useful:

- Copper
- Selenium
- (PGE)
- Silver
- Indium
- Tin
- Gold
- etc.



Analysis of large samples (5 g up to tens of kg



Potentially Suitable analytical methods

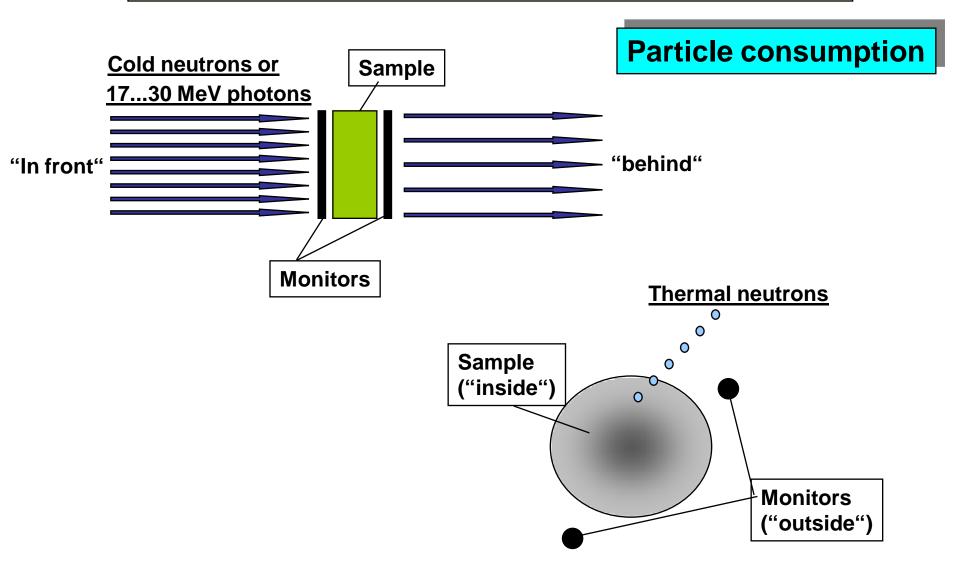
X-ray fluorescence analysis, continuous

Activation analysis

- with photons (PAA)
- with neutrons (NAA)



PHOTON BEAM GEOMETRY vs. REACTOR GEOMETRY





PHOTON BEAM GEOMETRY vs. REACTOR GEOMETRY

Particle consumption

Cold neutrons or photons

 Linear (exponential) interpolation possible **Thermal neutrons**

Extrapolation from "outside" to "inside"

Flux depression and self-shielding

• Matrix Z dependent

• Strongly matrix dependent







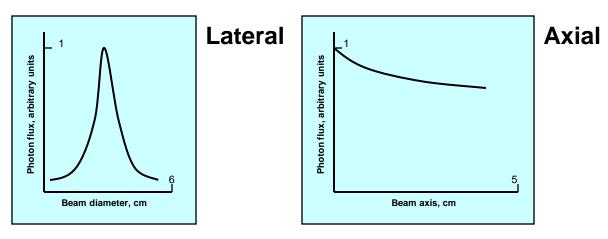
Cold neutrons or photons

Thermal neutrons

 \mathbf{O}

 lateral: high axial: low radial: high circumferential: low

Example: 30 MeV bremsstrahlung



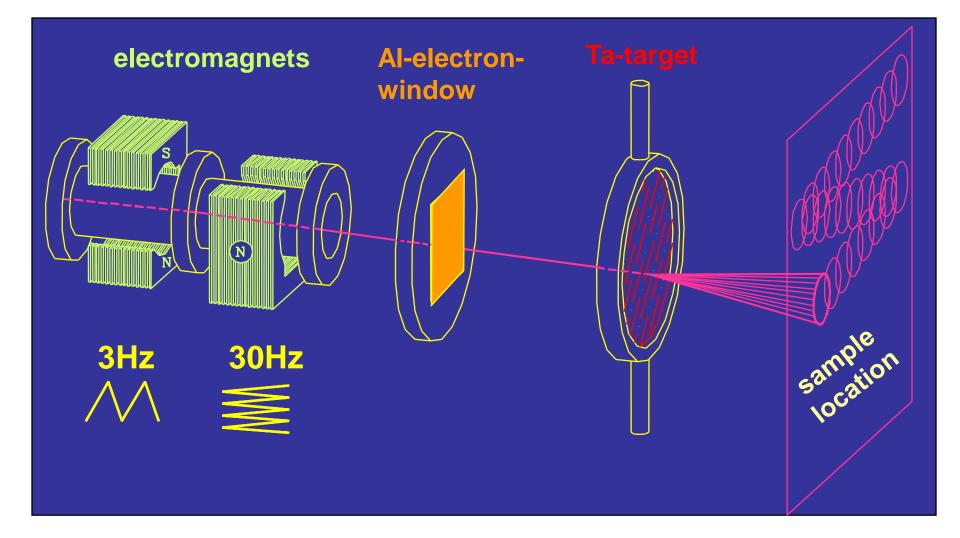
FWHM ca. 1.5cm

Ca. 18% flux decrease @ 4.5cm



THE SOLUTION:

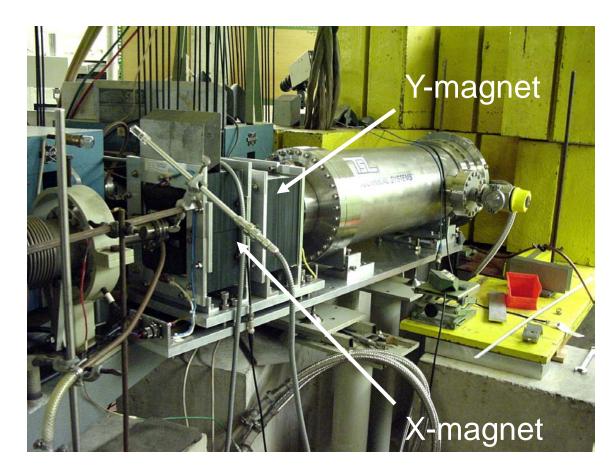
High Energy Photon Screen (HEPS)





6

End of beamline with x,y-scanning magnets



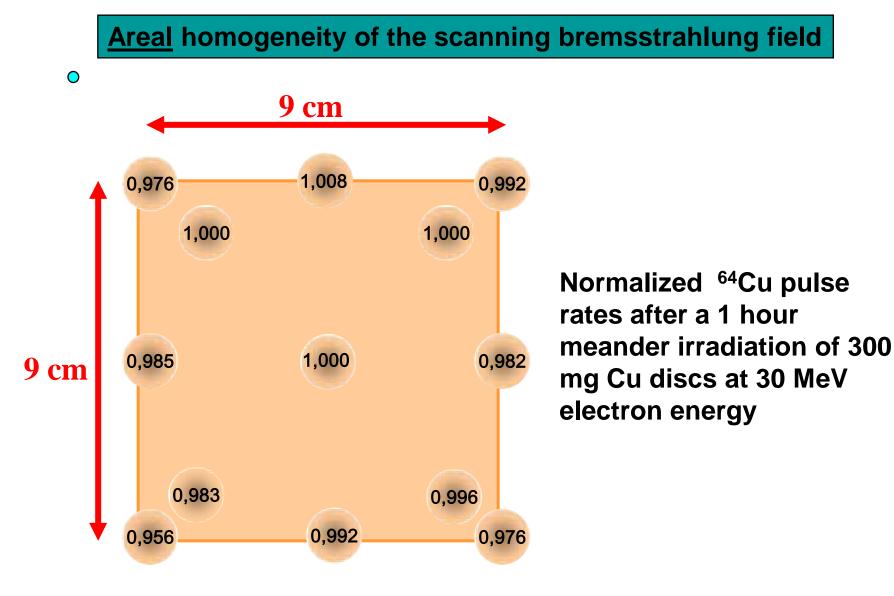


 ${\circ}$

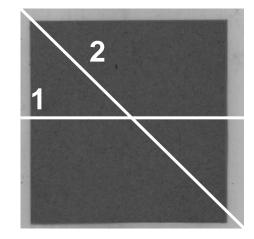
<u>Areal</u> homogeneity of the scanning bremsstrahlung field



Copper sheet matrix



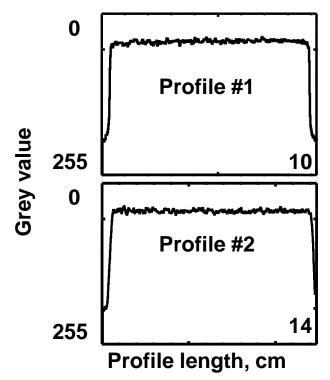
B



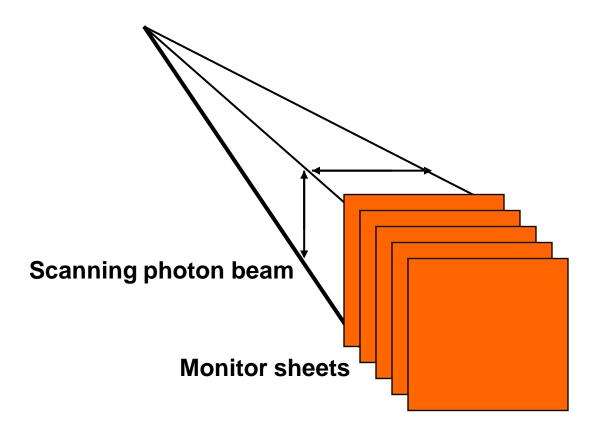
• <u>Areal</u> homogeneity of the scanning bremsstrahlung field

Autoradiographic approach

By

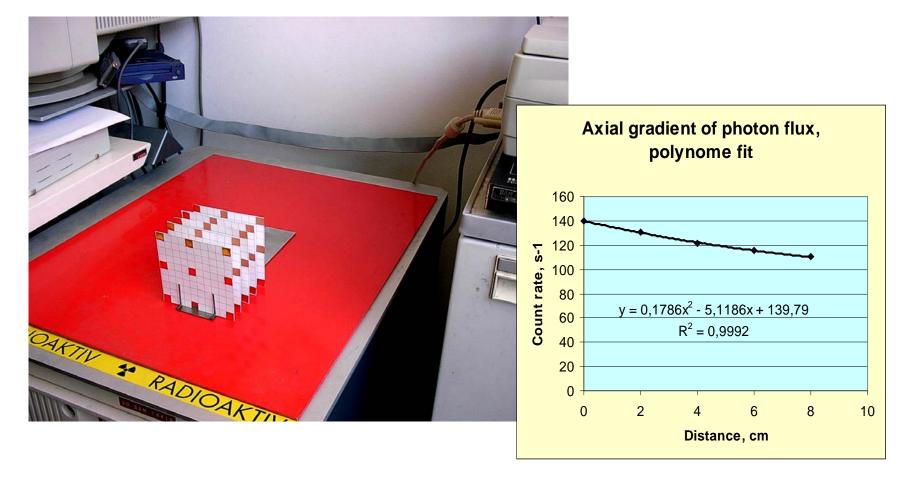


• <u>Axial</u> homogeneity of the scanning bremsstrahlung field



BAM

• <u>Axial</u> homogeneity of the scanning bremsstrahlung field





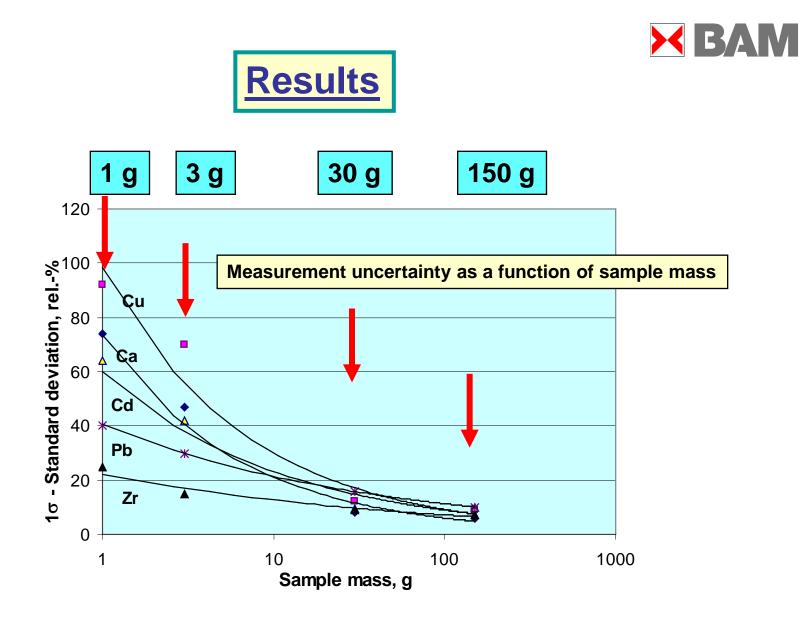


Contents, mg/kg (± 1 o , rel.%)			
Content	<u>PAA</u> ¹ 4.8 (16) 652 (6) 183 (13) 31 (26)	NAA ² 4.6 (3.0) n.d. ³ 48.8 (9.2) n.d. 48.8 (9.2) n.d. 5.36 (8.3) 0.162 (3.8) n.d. 25.7 (8.4) n.d.	
Tin% Antimony% Barium% Cerium% Gold Lead% Uranium	0.127 (5)	0.122 (35) 0.247 (1.1) 4.5 (2) 0.218 (2.8) 1.91 (2.1) n.d. n.d.	

¹Average out of 5 single samples @ 150 g

²One sample @ 1.22 kg

³n.d.: not detected









Large samples can be analysed





Improved representativeness

Example: Isotope Production

Why use Photon Activation Methods?

Why is it better or worse than reactor methods?

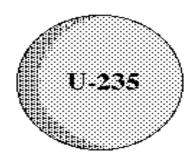
How does it work?

Is it practical?

Current reactor production technique:

fission in a reactor produces a variety of isotopes (about 6% of the time it makes 99Mo).





n° + ²³⁵U \implies ⁹⁹Mo + Sn + x n^o (~6%)

^{99m}Tc – metastable Technetium-99

 Nice thing about ⁹⁹Mo – it has a half life of 2.7 days and can be shipped around.

- It decays to ^{99m}Tc which has a half life of 6 hours. This is used on patients.
 Approximately <u>1.6M North Americans per</u> <u>month</u> get a 'treatment'.
- ^{99m}Tc is extracted from ⁹⁹Mo by "milking the cow".

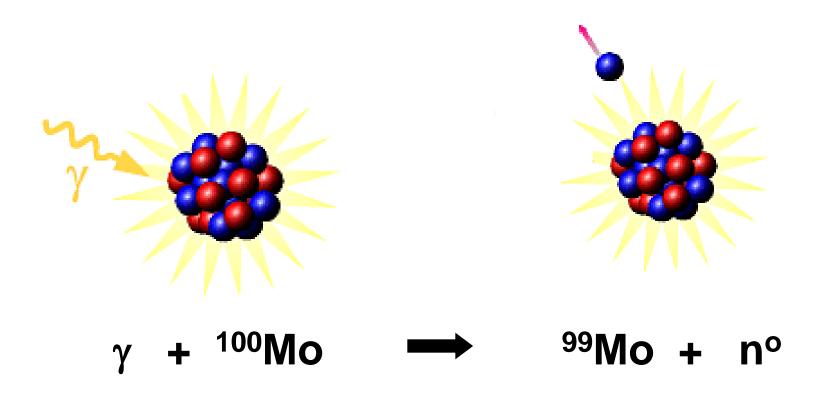
Problems with current technique

High level waste

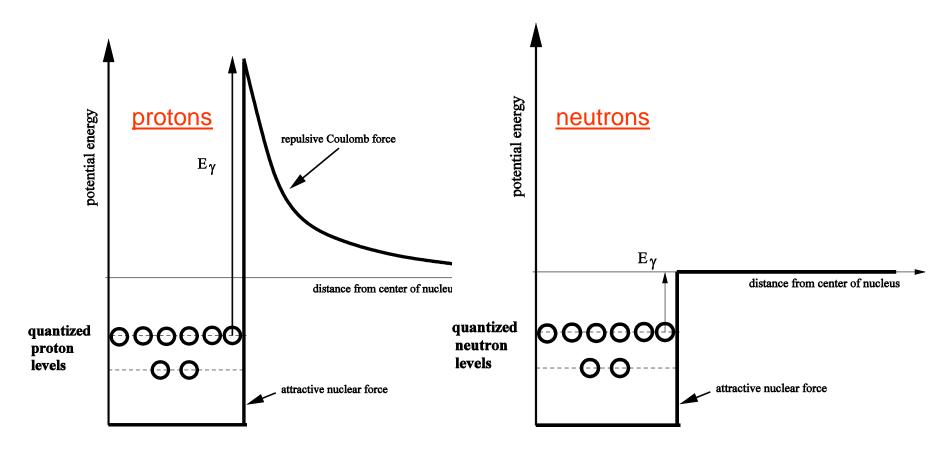
- Worldwide, 10's of kilograms of (HEU) HLW waste each year.
- <u>Nuclear proliferation</u>
- U.S. exports 15.5 kg of HEU (highly enriched uranium) to Canada each year.
- > 93% enrichment HEU made for nuclear warheads.
- Reliability of supply
- 2007 supply crisis.
- Future problems importing radioactive materials.

(y,n) production technique

- Photon excites ¹⁰⁰Mo nucleus.
- Knocks out a neutron leaving ⁹⁹Mo.



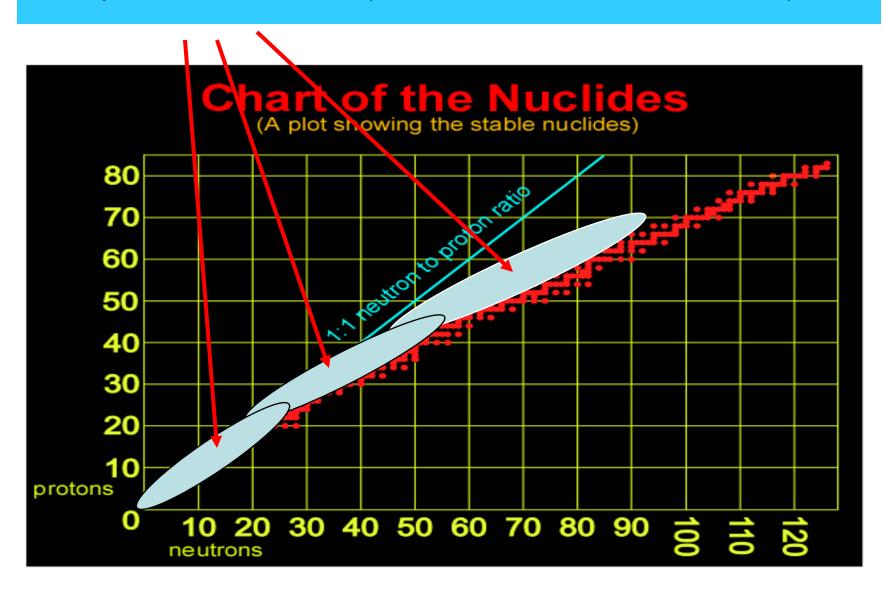
In general, photons make *proton rich* isotopes



Coulomb energy barrier for protons ...

but not for neutrons.

(gamma, x) reactions efficiently populate moderately proton-rich nuclei (and some neutron-rich nuclei)



Isotope	Crude yield per 100 kW per week from 100g targets at 50 MeV electron beam energy
F-18*	9 kCi/wk
Cu-64*	10 kCi/wk
Ba-131*	15 kCi/wk
Mo-99*	2 kCi/wk
In-111*	2 kCi/wk
Y-88*	6 Ci/wk
Se-75*	6 Ci/wk

* - Taken from <u>measured yield at 2 kW</u> using IAC linac and scaled by power, mass, isotopic enrichment, geometric corrections, etc. Note that these rates are consistent with Segebade et al. and the Mo-99 estimates of TRIUMF.

So why hasn't Photon Activation been exploited?

In two words:

Specific Activity (or lack thereof)

This is the Achilles heel of Photon Activation.

So why hasn't Photon Activation been exploited?

Example: ⁹⁹Mo –

The maximum Specific Activity of ⁹⁹Mo in a post-gammairradiated pure ¹⁰⁰Mo target is in the 1-10 Ci/g range (see Bennett, 1998 or TRIUMF, 2008).

But if one separates the ⁹⁹Mo from a post-irradiated pure ¹⁰⁰Mo target, the answer is very different. Advances in materials science have enabled us to produce

better than 1 kCi/g specific activity of ⁹⁹Mo.

Issues and Questions for Scale-up:

- ISU experiments were done with 1-2 kW electron beams and 1 g targets: scale-up to 100 g targets at 100 or more kW electron beams raises significant questions about heat transfer, efficiency of separation, etc.
- Costs, optimization, target designs, multi-isotope production, FDA approvals, NRC licensing (of the material, not the accelerators), and many others all pose challenges.

IF high specific activity can be produced from Photon Activation, shouldn't this be exploited?

Advantages:

- No HEU, LEU or uranium of any kind → no safeguards or proliferation risks.
- 2) No HLW. And no major nuclear wastes of any kind.
- 3) No major decommissioning costs associated with accelerator facility (when compared to reactor decommissioning costs).
- 4) Such a facility could simultaneously produce a large number of isotopes that are currently in short supply.
- 5) Such a facility(s) would diversify isotope production, research and education capacity and help insure a reliable supply.

Limiting ourselves to photon activation, for the moment, what can we make?

<u>Via (γ, n):</u> 64Cu, 26Al, 131Ba/131Cs, 88Y, 197Hg, 192Ir, 203Hg, 99Mo, 75Se, 11C, 13N, 15O, 18F

<u>Via (γ, p):</u> 67Cu, 47Sc, 177Lu, 90Y, 110mAg, 166Ho, 57Co, 77Se

<u>Via (γ, 2n):</u> 77Br, 67Ga, 7Be, 111In

<u>Via (γ, np):</u> 82mRb, 64Cu

Thank You.

• Acknowledgements:

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